# ENDF/B-V NEUTRON CROSS SECTION EVALUATION FOR THE KRYPTON ISOTOPES 

A. Prince

January 1979

INFORMATION ANALYSIS CENTER REPORT

NATIONAL NUCLEAR DATA CENTER
brookhaven national laboratory
UPTON, NEW YORK 11973


# ENDF/B-V NEUTRON CROSS SECTION EVALUATION FOR THE KRYPTON ISOTOPES 

A. Prince

## 

January 1979

NATIONAL NUCLEAR DATA CENTER

BROOKHAVEN NATIONAL LABORATORY ASSOCIATED UNIVERSITIES, INC.

UNDER CONTRACT NO. EY-76-C-02-0016 WITH THE
UNITED STATES DEPARTMENT OF ENERGY

## NOTICE

This report was prepared as an account of work sponsored by the United States Government. Neither the United States nor the United States Department of Energy (DOE), nor any of their employees, nor any of their contractors, subcontractors, or their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness or usefulness of any information, apparatus, product or process disclosed, or represents that its use would not infringe privately owned rights.

Printed in the United States of America
Available from
National Technical Information Service
U.S. Department of Commerce

5285 Port Royal Road
Springfield, VA 22161
Price: Printed Copy $\$ 5.25$; Microfiche $\$ 3.00$
August 1979485 copies

## ABSTRACT

This report describes the evaluation of the neutron cross-section data for the six stable isotopes of Kr from $10^{-5} \mathrm{eV}$ to 20 MeV . These evaluations incorporate all the new data on these isotopes including those on the resonance parameters, ievel schemes of the various isotopes and residual nuclei and reaction data. Evaluation procedures adopted to assess experimental data and the nuclear model calculations used are described.

|  | CONTENTS | Page |
| :---: | :---: | :---: |
| 1.0 | Introduction | 1 |
| 2.0 | Resonance Region | 1 |
| 3.0 | Continuum Region Cross Section | 4 |
| 4.0 | Angular Distribution of Secondary Neutrons | 4 |
| 5.0 | Energy Distribution of Secondary Neutrons | 5 |
|  | Acknowleagments | 5 |
|  | References | 5 |
| TABLES |  |  |
| 1 | Adopted Resonance Parameters | 6 |
| 2 | Experimental and Recommended Thermal Capture Cross Sections for ${ }^{80} \mathrm{Kr}$. | 6 |
| 3 | Natural Abundance and Mass of Krypton Isotopes | 7 |
| 4 | Optical Model Potential | 7 |
| 5 | $Q$ Values for Nuclear Reactions | 7 |
| 6 (a) | Energy Levels for Hauser-Feshbach Calculations: ${ }^{78} \mathrm{Kr}$ | 8 |
| 6(b) | Energy Levels for Hauser-Feshbach Calculations: ${ }^{80} \mathrm{Kr}$ | 8 |
| 6(c) | Energy Levels for Hauser-Feshbach Calculations: ${ }^{82} \mathrm{Kr}$ | 9 |
| 6 (d) | Energy Levels for Hauser-Feshbach Calculations: ${ }^{83} \mathrm{Kr}$ | 9 |
| 6 (e) | Energy Levels for Hauser-Feshbach Calculations: ${ }^{84} \mathrm{KI}$ | 10 |
| 6 (f) | Energy Levels for Hauser-Feshbach Calculations: ${ }^{86} \mathrm{Kr}$ | 10 |


| FIG. | LIST OF FIGURES | Page |
| :---: | :---: | :---: |
| 1 | ${ }^{78} \mathrm{Kr}$ Total Neutron Cross Section in the Resonance Region | 11 |
| 2 | 78 Kr Elastic Neutron Cross Section in the Resonance Region | 11 |
| 3 | 78 Kr Capture Neutron Cross Section in the Resonance Region | 12 |
| 4 | ${ }^{80} \mathrm{Kr}$ Total Neutron Cross Section in the Resonance Region | 12 |
| 5 | ${ }^{80} \mathrm{Kr}$ Elastic Neutron Cross Section in the Resonance Region | 13 |
| 6 | ${ }^{80} \mathrm{Kr}$ Capture Neutron Cross Section in the Resonance Region | 13 |
| 7 | ${ }^{82} \mathrm{Kr}$ Total Neutron Cross Section in the Resonance Region | 14 |
| 8 | ${ }^{82} \mathrm{Kr}$ Elastic Neutron Cross Section in the Resonance Region | 14 |
| 9 | ${ }^{82} \mathrm{Kr}$ Capture Neutron Cross Section in the Resonance Region | 15 |
| 10 | ${ }^{83} \mathrm{Kr}$ Total Neutron Cross Section in the Resonance Region | 15 |
| 11. | ${ }^{83} \mathrm{Kr}$ Elastic Neutron Cross Section in the Resonance Region | 16 |
| 12 | $83^{\mathrm{Kr}}$ Capture Neutron Cross Section in the Resonance Region | 16 |
| 13 | ${ }^{84} \mathrm{Kr}$ Total Neutron Cross Section in the Resonance Region | 17 |
| 14 | ${ }^{84} \mathrm{Kr}$ Elastic Neutron Cross Section in the Resonance Region | 17 |
| 15 | ${ }^{84} \mathrm{Kr}$ Capture Neutron Cross Section in the Resonance Region | 18 |
| 16 | ${ }^{86} \mathrm{Kr}$ Total Neutron Cross Section in the Resonance Region | 18 |
| 17 | ${ }^{86} \mathrm{Kr}$ Elastic Neutron Cross Section in the Resonance Reqion | 19 |
| 18 | ${ }^{86} \mathrm{Kr}$ Capture Neutron Cross Section in the Resonance Region | 19 |
| 19 | Kr (Nat) Total Neutron Cross Section (0.002 to 10.0 eV ) | 20 |
| 20 | Kr (Nat) Total Neutron Cross Section (10.0 ev to 100.0 keV ) | 21 |
| 21 | Kr (Nat) Total Neutron Cross Section ( 0.1 to 20.0 MeV ) | 22 |
| 22 | ${ }^{78} \mathrm{Kr}\left(\mathrm{n}, \mathrm{n}^{\prime}\right)$ First Excited Neutron Cross Section $(Q=-455.0 \mathrm{keV})$ | 23 |
| 23 | ${ }^{78} \mathrm{Kr}(\mathrm{n}, \mathrm{n}$ ') Continuum Neutron Cross Section | 23 |
| 24 | ${ }^{78} \mathrm{Kr}$ Total Inelastic Neutron Cross Section | 24 |
| 25 | ${ }^{78} \mathrm{Kr}(\mathrm{n}, \mathrm{p})$ Neutron Cross Section | 24 |
| 26 | $78^{\mathrm{Kr}}(\mathrm{n}, \alpha)$ Neutron Cross Section | 25 |
| 27 | $78 \mathrm{Kr}(\mathrm{n}, 2 \mathrm{n})$ Neutron Cross Section | 25 |
| 28 | ${ }^{78} \mathrm{Kr}$ Non-elastic Neutron Cross Section | 26 |
| 29 | $80 \mathrm{Kr}(\mathrm{n}, \mathrm{n}$ ') First Excited Neutron Cross Section ( $\mathrm{Q}=-616.0 \mathrm{keV}$ ) | 26 |
| 30 | ${ }^{80} \mathrm{Kr}(\mathrm{n}, \mathrm{n}$ ') Continuum Neutron Cross Section | 27 |
| 31 | ${ }^{80} \mathrm{Kr}$ Total Inelastic Neutron Cross Section | 27 |
| 32 | ${ }^{80} \mathrm{Kr}(\mathrm{n}, \mathrm{p})$ Neutron Cross Section | 28 |
| 33 | ${ }^{80} \mathrm{Kr}(\mathrm{n}, \alpha)$ Neutron Cross Section | 28 |
| 34 | ${ }^{80} \mathrm{Kr}(\mathrm{n}, 2 \mathrm{n})$ Neutron Cross Section | 29 |
| 35 | $8^{0} \mathrm{Kr}$ Non-elastic Neutron Cross Section | 29 |
| 36 | ${ }^{82} \mathrm{Kr}(\mathrm{n}, \mathrm{n}$ ') First Excited Neutron Cross Section | 30 |
| 37 | ${ }^{82} \mathrm{Kr}\left(\mathrm{n}, \mathrm{n}^{\prime}\right)$ Continumm Neutron Cross Section | 30 |
| 38 | ${ }^{82} \mathrm{Kr}$ Total Inelastic Neutron Cross Section | 31 |
| 39 | ${ }^{82} \mathrm{Kr}(\mathrm{n}, \mathrm{p})$ Neutron Cross Section | 31 |
| 40 | $82^{\mathrm{Kr}}(\mathrm{n}, \alpha)$ Neutron Cross Section | 32 |
| 41 | ${ }^{82} \mathrm{Kr}(\mathrm{n}, 2 \mathrm{n})$ Neutron Cross Section | 32 |
| 42 | ${ }^{82} \mathrm{Kr}$ Non-elastic Neutron Cross Section | 33 |
| 43 | ${ }^{83} \mathrm{Kr}\left(\mathrm{n}, \mathrm{n}^{\prime}\right)$ Eirst Excited Neutron Cross Section ( $Q=-9.4 \mathrm{keV}$ ) | 33 |
| 44 | ${ }^{83} \mathrm{Kr}\left(\mathrm{n}, \mathrm{n}^{\prime}\right)$ Continuum Neutron Cross Section | 34 |
| 45 | ${ }^{83} \mathrm{Kr}$ Total Inelastic Neutron Cross Section | 34 |
| 46 | ${ }^{83} \mathrm{Kr}(\mathrm{n}, \mathrm{p})$ Neutron Cross Section | 35 |
| 47 | ${ }^{83} \mathrm{Kr}(\mathrm{n}, \alpha)$ Neutron Cross Section | 35 |
| 48 | ${ }^{83} \mathrm{Kr}(\mathrm{n}, 2 \mathrm{n})$ Neutron Cross Sextion | 36 |
| 49 | ${ }^{83} \mathrm{Kr}(\mathrm{n}, 3 \mathrm{n})$ Neutron Cross Section | 36 |

Figures (cont'd) Page
$50{ }^{83} \mathrm{Kr}$ Non-elastic Neutron Cross section ..... 37
${ }^{84} \mathrm{Kr}(\mathrm{n}, \mathrm{n}$ ') First Excited Neutron Cross Section ( $Q=-882.0 \mathrm{keV}$ ) ..... 37
${ }^{84} \mathrm{Kr}\left(\mathrm{n}, \mathrm{n}^{\prime}\right)$ Continuum Neutron Cross Section ..... 38
${ }^{84} \mathrm{Kr}$ Total Inelastic Neutron Cross Section ..... 38
${ }^{84} \mathrm{Kr}(\mathrm{n}, \mathrm{p})$ Neutron Cross section ..... 39
${ }^{84} \mathrm{Kr}(\mathrm{n}, \alpha)$ Neutron Cross Section ..... 39
${ }^{84} \mathrm{Kr}(\mathrm{n}, 2 \mathrm{n})$ Neutron Cross Section ..... 40
${ }^{84} \mathrm{Kr}$ Non-elastic Neutron Cross Section ..... 40
${ }^{86} \mathrm{Kr}\left(\mathrm{n}, \mathrm{n}^{\prime}\right)$ First Excited Neutron Cross Section ( $Q=-1.563 \mathrm{MeV}$ ) ..... 41
${ }^{86} \mathrm{Kr}(\mathrm{n}, \mathrm{n}$ ') Continuum Neutron Cross Section ..... 41
${ }^{86} \mathrm{Kr}$ Total Inelastic Neutron Cross Section ..... 42
${ }^{86} \mathrm{Kr}(\mathrm{n}, \mathrm{p})$ Neutron Cross Section ..... 42
${ }^{86} \mathrm{Kr}(n, 2 n)$ Neutron Cross Section ..... 43
${ }^{86} \mathrm{Kr}(\mathrm{n}, 3 \mathrm{n})$ Neutron Cross Section ..... 43
${ }^{86} \mathrm{Kr}$ Non-elastic Neutron Cross section ..... 44
Kr (Nat) Capture Neutron Cross Section $\left(10^{-5}\right.$ ev to 20.0 MeV$)$ ..... 45
Kr (Nat) Elastic Neutron Cross Section ( $10^{-5} \mathrm{eV}$ to 20.0 MeV ) ..... 46
Kr (Nat) Total Neutron Cross Section ( $10^{-5} \mathrm{eV}$ to 20.0 MeV ) ..... 47

### 1.0 INTRODUCTION

Because of the importance of krypton as a tag material for detecting and locating fuel failure in the FFTF, 1 the decision was made to upgrade the evaluation for the six stable isotopes for inclusion in ENDF/B-V. The major differences between the ENDF/B-IV 1 evaluation and the new one are related to the recent experimental resonance parameter data of Block et al. ${ }^{2}$ New data on the level schemes of the various isotopes and residual nuclei resulting from neutron interactions also prompted a more detailed treatment of the various reaction cross sections. The evaluation covers all significant possible neutron-induced reactions from $10^{-5}$ ev to 20.0 MeV .

The isotopes and their corresponding MAT numbers are given below for ENDF/B-V.

| Isotope |  | MAT |
| :--- | :--- | ---: |
|  |  |  |
| 78 Kr |  | 1330 |
| $80_{\mathrm{Kr}}$ |  | 1331 |
| 82 Kr |  | 1332 |
| 83 Kr |  | 1333 |
| 84 Kr |  | 1334 |
| 86 Kr |  | 1336 |

2.0 RESONANCE REGION

78 Kr MAT $\left(10^{-5} \mathrm{eV}\right.$ to 865.0 eV$)$
The recent experimental data of Block et al. . ${ }^{2}$ were used for calculating the cross section and resonances integral in this energy region. Block et al., concluded the ${ }^{78} \mathrm{Kr}$ and ${ }^{80} \mathrm{Kr}$ form a doublet around $\mathrm{E}=106.0 \mathrm{eV}$, an assumption that was made in ENDF/IV. In addition, since neither Block ${ }^{2}$ nor Mann and Watson ${ }^{3}$ could definitely assign the $640-\mathrm{eV}$ level to any particular isotope, although they designated the possibilities to be ${ }^{78} \mathrm{Kr},{ }^{80} \mathrm{Kr}, 1,3$ and a doublet in ${ }^{82} \mathrm{Kr},{ }^{2}$ this resonance was used in both 78 Kr and ${ }^{80} \mathrm{Kr}$.

A negative resonance at $E_{R}=-120.6 \mathrm{eV}$ produced a thermal capture cross section

$$
\sigma(0.0253 \mathrm{eV})=4.85 \mathrm{~b}
$$

which compares favorably with the experimental

$$
\sigma(0.0253 \mathrm{eV})=4.71 \pm 0.68 \mathrm{~b} .4,5
$$

There are no experimental data for the absorption resonance integral for $78_{\mathrm{Kr}}$. The calculated value, based on the resonance parameters given in Table 1 , is $I_{a b}\left(E_{\text {cutoff }}=0.5 \mathrm{eV}\right)=23.67 \mathrm{~b}$. Block et al., estimated that
$I=20.0 \pm 1.0 \mathrm{~b}$ between 20.0 and 1200.0 eV . Both these values are about $f^{a b}$ e times as large as the value quoted in Ref. 6 , the reason being that in Ref. 6 only the $640.0-e V$ resonance was used.

Figures 1 to 3 shows the calculated values of the total, scattering, and capture cross sections, in the resonance region, based on the single-level Breit-Wigner formalism. The potential scattering cross section was estimated
to be 7.00 b on the basis of several measurements ranging from $6.2 \pm 0.1 \mathrm{~b}^{7}$ to $7.61 \pm 0.04 \mathrm{~b}^{8}$ (also see Refs, 6 and 9) for Kr (Nat).
${ }^{80} \mathrm{Kr}$ MAT $1331\left(10^{-5} \mathrm{eV}\right.$ to 1.0 keV )
Use of the resonance parameters shown in Table 1 , where a bound level is located $E=-118.0 \mathrm{eV}$, yielded a thermal capture cross section $\sigma_{\mathrm{n} \gamma}=11.74 \mathrm{~b}$, which may be compared with the range of values given in Table 2. ${ }^{\mathrm{n} \gamma}$

The resonance integral calculated by using the parameters in Table 1 produced a value of $I_{\text {( }}(0.5-e v$ cutoff $)=68.6 \mathrm{~b}$, which, although somewhat higher than those recommended in Refs. 2,6 , and 10 , is preferred since the most recent quoted value ${ }^{2}$ is based on data in the region 20.0 to 1200.0 eV which clearly underestimate the evaluated value that goes from 0.05 eV to the MeV range.

The potential scattering cross section is taken to be the same as that of ${ }^{78} \mathrm{Kr}$.

The total, elastic, and capture cross sections in the resonance regions are shown in Figures 4 to 6 .
${ }^{82} \mathrm{Kr}$ MAT $1332\left(10^{-5} \mathrm{eV}\right.$ to 100.0 eV$)$
only one resonance at an energy of $\sim 40 \mathrm{eV}$ has been firmly established for ${ }^{82} \mathrm{Kr} .2,6$ Block et al., ${ }^{2}$ have concluded that there is a doublet belonging to ${ }^{82} \mathrm{Kr}$ around 640.0 eV , but, since they give no neutron width ( $\Gamma_{n}$ ), this resonance has been ignored.

The radiation width $\Gamma_{\gamma r}$ was determined by the method proposed by Malecky et al. ${ }^{11}$ in which it is assumed that the main contribution to $\Gamma_{\gamma r}$ is due to the electric dipole radiation; from a semi-empirical analysis it ${ }^{\gamma}$ may be estimated as

$$
\begin{equation*}
\Gamma_{\gamma r}=10.5 \mathrm{U} / \mathrm{A}\left(\mathrm{a}^{-1 / 2}\right)\left(1.0-0.01 \mathrm{I}^{2}\right) \mathrm{eV} \tag{1}
\end{equation*}
$$

where
$A=$ mass of nucleus,
$U=$ effective energy of excitation $\left(U=E_{B}-\Delta\right)$,
$E_{B}=$ binding energy of compound nucleus,
$\Delta^{B}=\delta(N)$ and $\delta(p)$ the pairing energies,
$I=$ spin of target nucleus, and
$a=$ single-particle state density parameter near the Fermi surface.
The relation between $a$ and $A$ is based on the analysis of cook et al., 12 where
$a / A=0.00917 s+0.142$, and
$S=S(N)+S(Z)$ shell correction parameter.
A value of $\Gamma_{\gamma r}=236 \mathrm{mV}$ was calculated from Eq. (1) for ${ }^{82} \mathrm{Kr}$.
In addition to the reported resonance at 39.8 eV , it was necessary to assume a bound level at -42.83 ev (Table 1) to yield a thermal capture cross section $\sigma_{\mathrm{n} \gamma}=30.17 \mathrm{~b}$. The experimental value leading to the metastable state in ${ }^{8}{ }^{n \gamma} \mathrm{Kr}$ of $\sigma_{\mathrm{n} \gamma}{ }^{m}=20.0 \pm 3.5 \mathrm{~b}$ has been reported in Ref. 4. The value calculated here is barely within the recommended value of
$\sigma_{n \gamma}(m+g)=45 \pm 15 \mathrm{~b} .6$ A value $\sigma_{n \gamma}=25.0 \mathrm{~b}$ has been estimated by Iijima. ${ }^{13}$
As with ${ }^{78} \mathrm{Kr}$ and ${ }^{80} \mathrm{Kr}$, the potential scattering cross section was chosen to be $\sigma_{\text {pot }}=7.0 \mathrm{~b}$.

The resonance integral calculated from the data in Table 1 is $I_{\gamma}=183.56 \mathrm{~b}$ ( $E_{\text {cutoff }}=0.5 \mathrm{eV}$ ). The recommended value calculated from a single resonance at 39.8 eV has been given as $200 \pm 40 \mathrm{~b} .{ }^{6}$ A calculated $I_{\gamma}=190.0 \mathrm{~b}$ where 13.7 b is due to the unresolved region was reported in Ref. 13.

The total, elastic, and capture cross sections for the resonance region are displayed in Figures 7 through 9.
${ }^{83} \mathrm{Kr}$ MAT 1333 ( $10^{-5}$ to 520.625 eV )
The resonance parameters given in Ref. 3 yielded a thermal capture cross section of the only 16 b , which is much too small compared with the various recommended values $200 \pm 30 \mathrm{~b}^{6}$ and $205 \mathrm{~b} .{ }^{13}$

Calculating the radiation width by Eq. (1) yielded $\Gamma_{\gamma r}=233.0 \mathrm{mV}$; assuming a bound level at -3.9 eV gives a thermal capture cross section $\sigma_{\mathrm{n} \mathrm{\gamma}}=207.67 \mathrm{~b}$.

The calculated resonance integral is $I_{\gamma}=188.65 \mathrm{~b}$, which is lower than the value $I_{\gamma}=230 \pm 30 \mathrm{~b}$ quoted in Ref. 6 but higher than the value $I_{\gamma}=170.2 \mathrm{~b}$ in Ref. 13.

As earlier, the potential scattering cross section is the same as that of Kr (Nat), namely 7.0 b .

Figures 10 through 12 show the total, elastic, and capture cross section from 1.0 eV to 1.0 keV .
${ }^{84} \mathrm{Kr}$ MAT 1334 ( $10^{-5}$ to 2.0 keV )
An unassigned resonance reported by Mann and Watson ${ }^{3}$ was assumed to be due to ${ }^{84} \mathrm{Kr}$ at an energy of 1625.0 eV , with a neutron width $\Gamma_{\mathrm{n}}=2.84 .0 \mathrm{mV}$ and $\Gamma_{\gamma}=226.0 \mathrm{mV}$ (Eq. 1). This resonance along with the other two low-lying resonances at 519.0 and 580.0 eV yielded a thermal capture cross section $\sigma_{n \gamma}(0.0253 \mathrm{eV})=0.0864 \mathrm{~b}$. Kondaiah et al., ${ }^{4}$ reported an experimental value of $0.09 \pm 0.013 \mathrm{~b}$ for the $4.4-\mathrm{h}{ }^{85} \mathrm{Kr}$ and a value of $0.042 \pm 0.004 \mathrm{~b}$ for the $10.74-y^{85} \mathrm{Kr}$. A recommended value of $\sigma_{\mathrm{n} \gamma}=0.130 \pm 0.014 \mathrm{~b}^{85_{\mathrm{Kr}}}{ }^{(\mathrm{m}+\mathrm{g})}$ is given in Ref. 6 , and $\sigma_{n \gamma}=0.16 \mathrm{~b}$ is reported in Ref. 14.

The calculated resonance integral of magnitude 3.27 b is in good agreement with the calculated value of $2.7 \pm 0.7 \mathrm{~b}$ reported in Ref. 6 , and in excellent agreement with $I_{\gamma}=3.54$ given in Ref. 14.

The total, elastic, and capture cross sections are given in Figures 13 to 15.

## ${ }^{86} \mathrm{Kr}$ MAT 1336 ( $10^{-5} \mathrm{eV}$ to 13 keV )

Since no resonance parameters have been experimentally verified for ${ }^{86} \mathrm{Kx}$, it was necessary to make certain assumptions. From Mann et al., ${ }^{3}$ there is an unassigned resonance at an energy of $\sim 2700 \mathrm{eV}$. Since the thexmal capture cross section and resonance integral for ${ }^{86} \mathrm{Kr}$ have been estimated to be very small (see following references) this resonance was taken to be due to this isotope.

On the basis of an estimated s-wave strength function, $s_{0}=0.5 \times 10^{-4}$, and assuming $\Gamma_{\gamma} / \Gamma_{n} \ll 1$, the resonance parameters given in Table 1 were derived. These parameters produced a thermal capture cross section equal to 0.0635 b , which is not too far from the value $0.06 \pm 0.02 \mathrm{~b}^{6}$ and is consistent with the estimate of $0.062 \mathrm{~b}{ }^{14}$ Again, a potential scattering radius of 7.0 was used.

The resonance integral was calculated to be 0.122 b , which may be compared with the calculated value of $0.03 \pm 0.03 \mathrm{~b}^{3}$, and 0.07 quoted in Ref. 14 (also see Ref. 13).

The resonance region cross sections are shown in Figures 16 to 18.
Kr (Nat) Data
The total cross sections for the Kr isotopes calculated by using the resonance parameters in Table 1 were weighted by the respective isotopic abundance (see Table 3) and combined to produce the $K r$ (Nat) data. These results are compared with experimental data in Figures 19 and 20.

### 3.0 CONTINUUM REGION CROSS SECTION

The cross sections (total, elastic, inelastic capture, n-proton, and $n-{ }^{4} \mathrm{He}$ ) in the keV to $20-\mathrm{MeV}$ region were calculated by using the code CERBERO-2. ${ }^{16}$ The sources for the various optical model potentials employed are given in Table 4. No experimental data are available for the total cross sections for the Kr isotopes, but data do exist for Kr (Nat).

The O.M. calculations for the various isotopes were weighted by their abundance (see Table 3) and combined to give a calculated $\sigma_{\mathrm{T}}$ for Kr (Nat). The experimental data are compared with these results in Figure ${ }_{21}$.

The $n, \gamma, n, n^{\prime}, n, p$, and $n-{ }^{4} H e$ reaction calculations were carried out in the Hauser-Feshbach formalism with width fluctuation correction and with the $Q$ values given in Table 5. The level schemes used in these calculations are given in Table 6 (these levels were taken from Ref. 22 unless otherwise stated). The calculations were then modified to take into account other threshold reaction cross section ( $n, 2 n, n, 3 n, n, d, n, t$ ) which were calculated for ENDF/IV (see Ref. 1).

Figures 22 through 67 show some of the more important reaction cross sections, up to 20.0 MeV .
4.0 ANGULAR DISTRIBUTION OF SECONDARY NEUTRONS

The angular distribution of the elastically scattefed neutrons was interpreted in terms of a Legendre polynomial fit using CHAD ${ }^{26}$ for File 4.

For the inelastically scattered neutrons and the $n, 2 n$ reactions, the distribution was assumed to be isotropic.

### 5.0 ENERGY DISTRIBUTION OF SECONDARY NEUTRONS

The energy distributions of neutrons due to the $n, 2 n, n, 3 n$, and continuum inelastic scattering were expressed in terms of a normalized probability distribution having an evaporation spectrum given by

$$
f\left(E \rightarrow E^{\prime}\right)=E^{\prime} / I e^{-E^{\prime} / \theta}
$$

where $I=$ normalization constant, and $\theta=$ nuclear temperature. The energy dependence of $\theta$ was formulated according to Gilbert and Cameron. ${ }^{27}$

## ACKNOWLEDGMENTS

The author would like to express his appreciation to the various members of NNDC who helped in this evaluation effort. In particular, the diligent assistance of Anthony Fuoco, Frances Scheffel, and Sol Pearlstein, who were very supportive in carrying out many of the model calculations and plotting the various graphs, is gratefully acknowledged. In addition, a word of thanks is in order to Prof. R.C. Block and his colleagues at RPI for providing the resonance parameter data prior to publication and to R.E. Schenter (HEDL) for bringing this information to my attention. Finally, an expression of gratitude is extended to $F$. Fabbri and G. Reffo (CNEN) for furnishing their code CERBERO.

## REFERENCES

1. A. Prince, BNL-NCS-50503, 1974.
2. R.C. Block et al., Int. Conf., Harwell, England, 1978.
3. D.P. Mann and W.W. Watson, Phys. Rev. 116, 1516 (1959).
4. R. Kondaiah et al., Nucl. Phys. Al20, 329 (1968).
5. F.W. Walker et al., G.E. Chart of Nuclides (12th ed.) 1977.
6. S.F. Mughabghab and D.I. Garber, BNL 325 (3rd ed.), Vol. 1, 1973.
7. D.C. Rorer et al., Nucl. Phys. Al33, 410 (1969).
8. V.E. Krohn and G.R. Ringo, Phys. Rev. 148, 1303 (1966).
9. S.J. Cocking, J, Nucl. Eng. 6, 113 (195B),
10. J.G. Bradley and W.H. Johnsoñ, NSE 47, 151 (1972).
11. H. Malecky et a1., Sov. J. Nucl. Phỹs. 13, 133 (1971).
12. J.L. Cook, Austr. J. Phys. 20, 477 (1967).
13. S. Iijima et al., JAERI $120 \overline{6}$ (1971).
14. H. Saketa et al., JAERI, 11941970.
15. R. Genín et al., J. Phys. Radium 24, 21 (1963).
16. F. Fabbri et al., CERBERO-2, RT/F1(77)6, 1977.
17. D. Wilmore and P.E. Hodgson, Nucl. Phys. 55, 673 (1964).
18. F.D. Becchetti and G.W. Greenlees, Phys. Rev. 182, 1190 (1969).
19. M. Makowska et al., Rpt. 735/PL, Inst. Nucl. Phys., Cracow, Poland, 1970.
20. V.P. Vlasenko et al., Sov. Prog. Nucl. Phys., 206 (1961).
21. F.J. Vaughn et al., Phys. Rev. 118, 683 (1960).
22. Nuclear Data Sheets, Series B, Vol. 15, Academic Press, New York, 1975.
23. B.K. Arora at al., Phys. Rev. Clo, 2301 (1974).
24. S. Väisalä et al., Phys. Rev. $\overline{\mathrm{Cl3}}, 372$ (1976).

References (cont'd)
25. E.B. Flynn et al., Phys. Rev. Cl3, 568 (1976).
26. R.F. Berland, CHAD, NAA-SR-1123I (1965).
27. A. Gilbert and A.G.W. Cameron, Can. J. Phys. 43, 1446 \{1965\}.



| Table 6(a) <br> Energy Levels for Hauser-Feshbach Calculations: ${ }^{78} \mathrm{Kr}$ |  |  |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
| ${ }^{79} \mathrm{KX}$ |  |  |  |  |  | ${ }^{75} \mathrm{Sc}$ |  |
| $\sigma_{n \gamma}$ |  | $\sigma_{n n}$ |  |  |  | $\sigma_{\mathrm{n} \alpha}$ |  |
| 0.000 | $1 / 2^{+}$ | 0.000 | $\mathrm{O}_{+}^{+}$ | 0.000 | $1_{-}^{+}$ | 0.000 | $5 / 2^{+}$ |
| 0.300 | 7/2+ | 0.455 | $2_{+}^{+}$ | 0.032 | $2{ }^{-}$ | 0.110 | $3 / 2^{+}$ |
| 0.147 | 5/2- | 1.119 | $4_{+}^{+}$ | 0.180 | $4^{+}$ | 0.285 | 1/2+ |
| 0.183 | 3/2 ${ }^{-}$ | 1.978 | $6^{+}$ |  |  | 0.420 | 5/2 ${ }^{+}$ |
| 0.291 | 5/2 |  |  |  |  | 0.610 | 3/2 |
| 0.384 | 3/2- |  |  |  |  | 0.750 | 7/2 |
| 0.402 | 5/2- |  |  |  |  | 0.860 | 5/2- |
| 0.450 | 7/2+ |  |  |  |  | 0.903 | 3/2- |
| 0.534 | 1/2 ${ }^{+}$ |  |  |  |  | 1.030 | $7 / 2^{-}$ |
| 0.660 | 5/2- |  |  |  |  |  |  |
| 0.689 | $5 / 2^{+}$ |  |  |  |  |  |  |
| 0.695 | 3/2 ${ }^{+}$ |  |  |  |  |  |  |
| 0.720 | 7/2- |  |  |  |  |  |  |
| 0.810 | 3/2- |  |  |  |  |  |  |
| 1.038 | 7/2- |  |  | , |  |  |  |
| 1.912 | $1 / 2^{+}$ |  |  |  |  |  |  |
| 2.060 | $3 / 2^{+}$ |  |  |  |  |  |  |
| 2.768 7/2 ${ }^{+}$ |  |  |  |  |  |  |  |
| Table 6(b) <br> Energy Levels for Hauser-Feshbach Calculations: ${ }^{80} \mathrm{Kx}$ |  |  |  |  |  |  |  |
| ${ }^{81} \mathrm{Kr}$ |  | $80_{\mathrm{Kr}}$ Residual Nucleus: ${ }^{80}{ }_{\mathrm{B}}$ |  |  |  | ${ }^{77} \mathrm{Sc}$ |  |
| $\sigma_{n \gamma}$ | $I^{\text {T}}$ | $\sigma_{\mathrm{nn}}{ }^{\prime}$ | $I^{\top}$ | $\begin{gathered} \sigma_{\mathrm{np}} \\ \mathrm{E}(\mathrm{MeV}) \end{gathered}$ | $I^{\pi}$ | $\sigma_{\mathrm{na}}$ |  |
| 0.000 | $7 / 2^{+}$ | 0.000 | ${ }^{+}+$ | 0.000 | $\mathrm{I}_{-}^{+}$ | 0.000 | 1/2- |
| 0.190 | 1/2- | 0.616 | ${ }^{2+}$ | 0.037 | $2^{-}$ | 0.162 | $7 / 2^{+}$ |
| 0.457 | $3 / 2^{+}$ | 1.252 | $2+$ | 0.085 | $5{ }^{-}$ | 0.175 | $9 / 2^{+}$ |
| 0.549 | $1 / 2^{+}$ | 1.320 | ${ }^{+}$ | 0.256 | $2+$ | 0.239 | $3 / 2^{-}$ |
| 0.608 | 5/2- | 1.436 | ${ }^{4+}$ | 0.271 | $2-$ | 0.250 | 5/2- |
| 0.636 | $7 / 2^{-}$ | 2.390 | $6_{+}^{+}$ | 0.281 | $3+$ | 0.440 | 5/2 |
| 0.701 | $3 / 2^{+}$ | 3.400 | $8^{+}$ | 0.314 | $1+$ | 0.521 | $3 / 2_{+}$ |
| 0.919 | $3 / 2_{+}^{-}$ |  |  | 0.366 | $1-$ | 0.680 | $5 / 2^{+}$ |
| 1.025 | $3 / 2_{+}^{+}$ |  |  | 0.381 | $3{ }^{-}$ | 0.818 | $1 / 2^{-}$ |
| 1.108 | $1 / 2^{+}$ |  |  | 0.468 | ${ }^{+}$ | 0.912 | $3 / 2^{+}$ |
| 1.280 | $3 / 2^{+}$ |  |  | 0.684 | ${ }^{-}$ | 0.956 | $1 / 2^{+}$ |
| 1.677 | $5 / 2^{+}$ |  |  | 0.722 | $1_{+}^{+}$ | 1.006 |  |
| 1.803 | $3 / 2^{+}$ |  |  | 0.760 | $1+$ | 1.126 | $1 / 2^{+}$ |
|  |  |  |  | 0.835 | $\mathrm{I}_{+}^{-}$ | 1.412 | 1/2 |
|  |  |  |  | 1.139 | $1_{-}^{+}$ | 1.623 | 1/2 |
|  |  |  |  | 1.200 | ${ }^{2}+$ | 1.819 | 1/2- |
|  |  |  |  | 1.410 | $3{ }^{+}$ | 2.393 | $3 / 2^{-}$ |
|  |  |  |  | 1.700 | $2{ }^{-}$ |  |  |
|  |  |  |  | 1.880 | 2 |  |  |



| Table 6(e) <br> Energy Levels for Hausex-Feshbach Calculations: ${ }^{84} \mathrm{Kr}$ |  |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: |
| ${ }^{85} \mathrm{Kr}$ | ${ }^{84} \mathrm{Kr} \text { Residual Nucleus: }{ }_{84}{ }^{\text {Br }}$ |  |  |  | ${ }^{81} \mathrm{Sc}$ |  |
| $E(\mathrm{MeV}){ }^{\sigma^{n} \gamma} I^{\pi}$ | $\begin{gathered} \sigma_{\mathrm{nn}} \\ \mathrm{E}(\mathrm{MeV}) \end{gathered}$ | $I^{\pi}$ | $\underset{(\mathrm{MeV})}{\sigma_{\mathrm{np}}}$ | $I^{\prime \prime}$ | $\mathrm{E}\left({ }^{(\mathrm{MeV})}{ }_{\mathrm{n} \alpha}\right.$ | $I^{\prime \prime}$ |
| 0.000 9/2 ${ }^{+}$ | 0.000 | $\mathrm{O}_{+}^{+}$ | 0.000 | $2{ }^{-}$ | 0.000 | $1 / 2^{-}$ |
| $0.3051 / 2^{-}$ | 0.822 | $2_{+}^{+}$ | 0.408 | $1-$ | 0.103 | 7/2+ |
| 1.050 3/2- | 1.834 | $\mathrm{O}_{+}^{+}$ | 0.557 | 5 | 0.294 | 9/2 ${ }^{+}$ |
| 1.230 5/2 ${ }^{-}$ | 1.900 | $2_{+}^{+}$ | 0.873 | 2 | 0.468 | 1/2 |
| 1.767 3/2 ${ }^{+}$ | 2.086 | ${ }_{+}^{+}$ | 0.979 | 3 | 0.625 | 5/2 |
| $2.108 \mathrm{l} 2^{-}$ | 2.337 | $4^{+}$ | 1.196 | $4^{-}$ | 1.053 | $3 / 2^{+}$ |
| $2.166 \mathrm{l} / 2^{+}$ | 2.705 | $3-$ |  |  | 1.232 | 1/2+ |
| 2.237 7/2 ${ }^{+}$ | 2.775 | $2^{+}$ |  |  | 1.303 | $3 / 2^{+}$ |
| 2.418 5/2- | 3.048 | ${ }_{4}^{+}$ |  |  | 1.406 | $1 / 2+$ |
| 3.380 7/2 | 3.225 | $1-$ |  |  | 1.725 | $3 / 2+$ |
|  | 3.477 | $1-$ |  |  | 1.828 | $5 / 2^{+}$ |
|  | 3.570 | 3 |  |  | 2.174 | $3 / 2^{+}$ |
|  | 3.650 | $5^{-}$ |  |  | 2.550 | $5 / 2^{+}$ |
|  | 3.721 | $3^{-}$ |  |  | 2.680 | $1 / 2^{+}$ |
|  |  |  |  |  | 3.070 | $3 / 2+$ |
|  |  |  |  |  | 3.310 | $5 / 2+$ |
|  |  |  |  |  | 3.490 | $3 / 2+$ |
|  |  |  |  |  | 3.570 | $1 / 2+$ |
|  |  |  |  |  | 3.670 3.720 | $3 / 2+$ $5 / 2+$ |

*Reference 23.

| Table 6(f) <br> Energy Levels for Hauser-Feshbach Calculations: ${ }^{86} \mathrm{Kr}$ |  |  |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
| ${ }^{87}$ |  | Residual Nucleus: ${ }^{86} \mathrm{Kr}$ |  |  |  | ${ }^{83} \mathrm{Sc}$ |  |
| ${ }^{(M e V)}{ }^{\sigma_{n}}$ | $I^{T 1}$ | $\mathrm{E(MeV)}{ }_{\mathrm{nn}}{ }^{\sigma^{\prime}}$ | $I^{\top}$ | $\mathrm{E} \stackrel{\sigma_{n p}}{(\mathrm{MeV})}$ | $I^{\pi}$ | E (MeV) | $I^{\top}$ |
| 0.000 | $5 / 2^{+}$ | 0.000 | $\mathrm{O}_{+}^{+}$ | 0.000 | 2 | 0.000 | $9 / 2^{+}$ |
| 0.529 | $1 / 2^{+}$ | 1.563 | $2_{+}^{+}$ | 0.196 | $1{ }^{-}$ | 0.220 | 1/2- |
| 1.468 | $3 / 2^{+}$ | 2.248 | $4_{+}^{+}$ | 0.268 | 4 | 1.050 | 3/2 |
| 1.873 | $5 / 2^{+}$ | 2.355 | $2_{+}^{+}$ | 0.390 | 2 | 1.230 | $5 / 2+$ |
| 1.996 | $3 / 2^{+}$ | 2.733 | $\mathrm{O}_{-}^{+}$ | 0.862 | $1{ }^{-}$ | 1.767 | 3/2 ${ }^{+}$ |
| 2.080 | $5 / 2^{+}$ | 3.109 | ${ }^{-}$ | 1.210 | ${ }_{1}^{+}$ | 2.108 | $1 / 2+$ |
| 2.112 | $3 / 2_{-}^{+}$ | 3.542 | ${ }_{+}^{+}$ | 2.232 | ${ }_{1+}^{+}$ | 2.166 | $1 / 2+$ |
| 2.250 | 9/2 ${ }^{+}$ | 3.832 | ${ }^{+}$ | 2.392 | ${ }_{1}^{+}$ | 2.237 | 7/2 ${ }^{-}$ |
| 2.277 | $1 / 2^{+}$ | 3.959 | ${ }_{4}^{+}$ | 2.772 | 1 | 2.418 | 5/2- |
| 2.515 | 7/2+ | 4.111 | $2_{+}^{+}$ |  |  | 3.380 | 7/2 |
| 2.775 | $3 / 2^{+}$ | 4.194 | 2 |  |  |  |  |
| 2.825 | $3 / 2_{+}^{+}$ | 4.298 | $3+$ |  |  |  |  |
| 3.015 | $5 / 2^{+}$ | 4.668 | ${ }_{+}^{+}$ |  |  |  |  |
| 3.223 | $3 / 2^{+}$ | 4.826 | ${ }^{2+}$ |  |  |  |  |
| 3.237 | $5 / 2^{+}$ | 4.948 | 2 |  |  |  |  |

*References 23 and 25.


Figure 1.


Figure 2.


Figure 3.


Figure 4.

- 12 -


Figure 5.


Figure 6.


Figure 8.


Figure 9.
${ }^{83} \mathrm{Kr}$ TOTAL
NEUTRON CROSS SECTION


Figure 10.

- 15 -


Figure 11.


Figure 12.

- 16 -


Figure 13.


Figure 14.


Figure 15.


Figure 16.


Figure 17.


Figure 18.




- 22 -


Figure 22.


Figure 23.


Figure 24.


Figure 25.

- 24 -


Figure 26.


Figure 27.


Figure 28.


Figure 29.


Figure 30.


Figure 31.

- 27 -


Figure 33.


Figure 34.


Eigure 35.


Figure 37.

- 30 -


Figure 38.


Figure 39.

- 31 -


Figure 40.


Figure 41.

- 32 -


Figure 42.


Figure 43.


Figure 44.


Figure 45.


Figure 47.

- 35 -


Figure 48.


Figure 49.


Figure 50.


Figure 51.


Figure 52.


Figure 53.

- 38 -


Figure 54.
$84 \mathrm{Kr}(n, a)$


Figure 55.

- 39 -


Figure 56.


Figure 57.

- 40 -


Figure 58.


Figure 59.


Figure 60.


Figure 61.


Figure 62.


Figure 63.


Figure 64

- 44 -

- 45 -


- 47 -

